3. RECYCLED URANIUM MASS FLOW

3.1 Uranium Recycle Description

A diagram to illustrate the RU material mass flow at PORTS is shown in Figure 3.1-1. The total quantity received from each source and the total quantity shipped to each receiver is shown. The center of the diagram represents the several processes the RU material may have passed through after receipt. There is a large difference between the quantity received and the quantity shipped. This difference is due to the diluting nature of the processes at PORTS. When RU is fed to the cascade, the quantity fed is only a small fraction of the total amount of uranium present in the cascade. Once fed, the RU is mixed with the other material already present, and can no longer be tracked based upon the original uranium content. Each constituent is separated from the original uranium and follows a different path through the PORTS cascade and other facilities. Therefore, PORTS tracks RU only until it loses it's unique identity; from that point, PORTS tracks each constituent of TRU and ⁹⁹Tc individually to show the constituents' mass flow and to perform a mass balance.

Recycled uranium was first introduced at PORTS in FY 1955 as UF₆ feed manufactured at Paducah from UO₃ received from Hanford (HRT) and Savannah River (SRT) reactor tails. Also in FY 1955 PPF was provided for PORTS feed. The PPF was contaminated with ⁹⁹Tc at an estimated 1 ppm (Ref. 2). The UO₃ from HRT/SRT was contaminated with Np, Pu, and ⁹⁹Tc at an estimated 0.24 ppm, 4 ppb, and 7 ppm, respectively, prior to FY 1967 and 0.09 ppm, 2.2 ppb, and 7 ppm, respectively, thereafter (Ref. 2). After feed was manufactured from the HRT/SRT oxide it was contaminated with Np, Pu, and ⁹⁹Tc at an estimated 0.18 ppm, 0.04 ppb, and 6.65 ppm, respectively, prior to FY 1967 and 0.068 ppm, 0.021 ppb, and 6.65 ppm, respectively, thereafter (Refs. 2 and 2a).

To illustrate and track the movement of RU, TRU and ⁹⁹Tc through PORTS, four campaigns which cover all significant events at PORTS from startup in FY 1955 through March 31, 1999 were developed. Each campaign addresses a specific grouping of RU for a specific time period.

The Depleted Reactor Tails – Campaign #1 (Figure 3.1-2), addresses feed manufactured from HRT/SRT oxide and PPF from FY 1955 through FY 1967. The Depleted Reactor Tails – Campaign #2 (Figure 3.1-3) addresses feed manufactured from HRT/SRT oxide and PPF from FY 1968 through March 31, 1999. Note: The bars which extend beyond FY 1978 are assumed to remain constant through March 1999. The Non-UF $_6$ RU Program – Campaign #3 (Figure 3.1-4) deals with RU of all forms of uranium at PORTS other than UF $_6$. These campaigns do not include 4.6 MTU of non-UF $_6$ potentially utilized for development activities in FY 1957. The remaining RU is captured in the UF $_6$ feed as Miscellaneous Cascade Feed - Campaign #4 (Figure 3.1-5).

Each campaign shows what is known, estimated or projected regarding RU. Each figure identifies the source of the RU, year(s) received at PORTS, quantity of RU, which process(es) the RU, TRU and ⁹⁹Tc passed through, and when the material was shipped from PORTS. Significant events that occurred during the period are shown. This method allows for a tabulation of the TRU and ⁹⁹Tc by year to provide a year-end inventory, and establishes the RU constituent inventory as of March 31, 1999.

The RU, containing TRU and ⁹⁹Tc, was first introduced between FY 1955 and FY 1958 when approximately 527 MTU of feed manufactured from HRT/SRT oxide was received. This material is estimated to have contained a total of 95g Np, 0.021g Pu, and 3.7kg ⁹⁹Tc. Also, Paducah feed was utilized beginning in FY 1955 and continues to the present time. Between FY 1955 and FY 1971, ⁹⁹Tc was present at a concentration of approximately 1 ppm. During this time, 43.5 kg of ⁹⁹Tc is estimated to have been fed into the PORTS cascade. To establish the annual inventory of ⁹⁹Tc from Paducah feed, the total quantity received during this period was distributed evenly over the 17-year period. Various sections of this report discuss in detail specific plant facilities which processed/concentrated RU, TRU, and ⁹⁹Tc and will not be repeated here except as required to describe the flow of the RU and its constituents through each campaign.

3.1.1 Campaign #1

During Campaign #1, the TRU and ⁹⁹Tc contaminants that entered the cascade with feed manufactured from HRT/SRT oxide and early PPF were substantially removed during the first cascade change-out program. It is assumed that during this period, material was fed upon receipt and the empty cylinders with heels were returned to Paducah/Oak Ridge. These cylinders were not cleaned at PORTS; therefore, any TRU, RU, and ⁹⁹Tc contained in the heels went to Paducah or Oak Ridge.

Figures 3.1-1

PORTS RU MATERIAL FLOW THROUGH

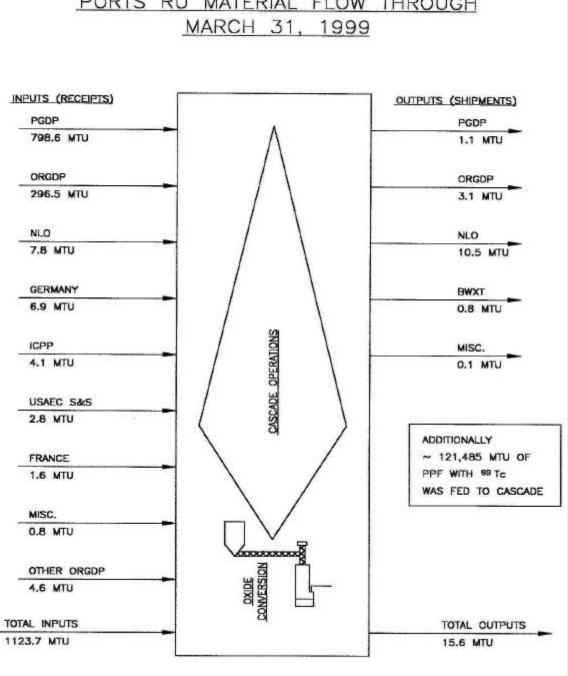


Figure 3.1-2

Figure 3.1-3

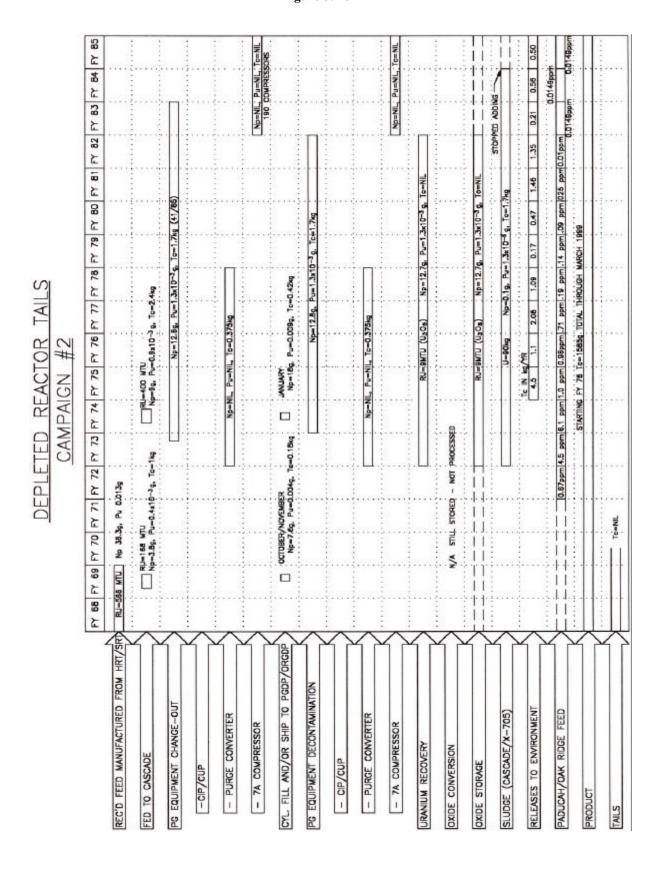


Figure 3.1-4

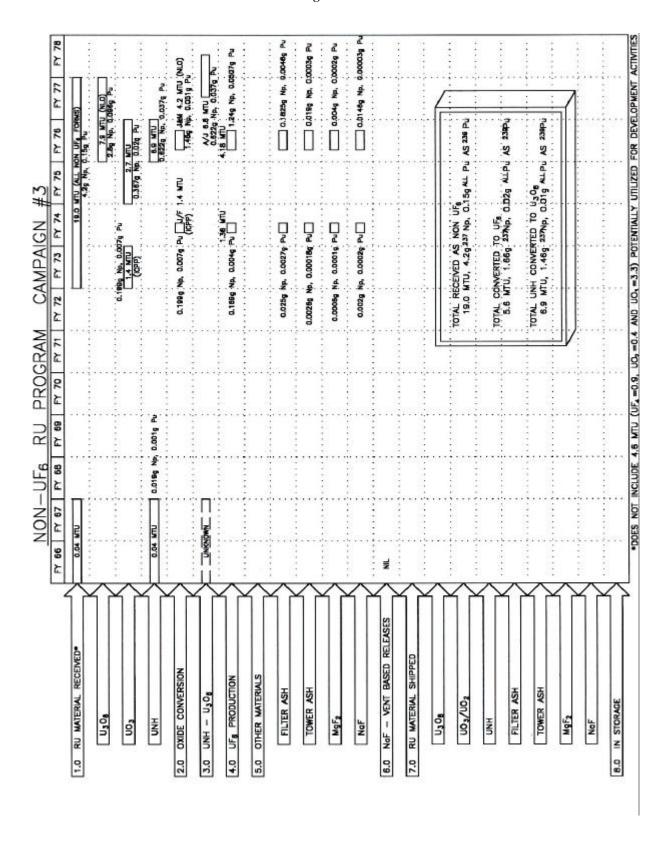
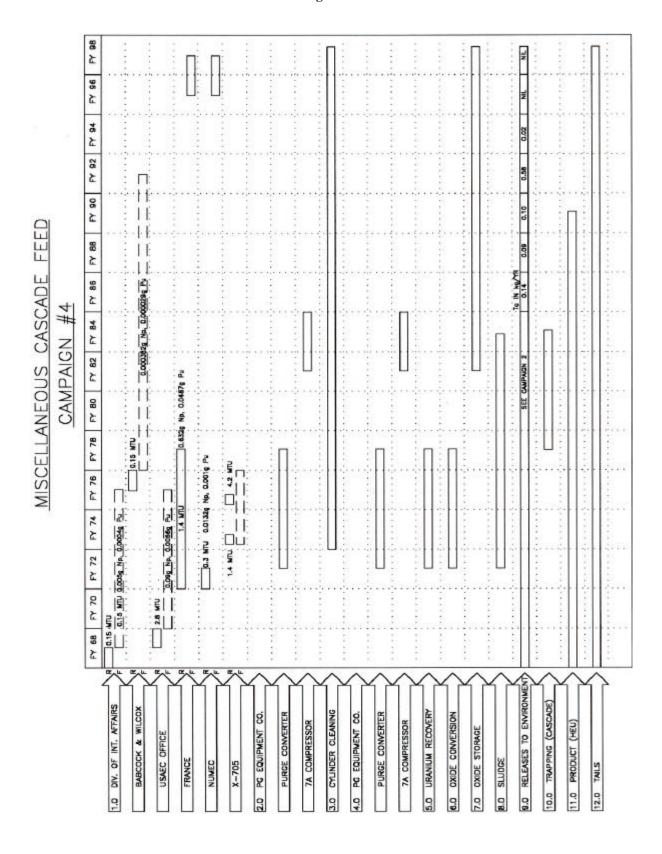


Figure 3.1-5



When RU/FP material was fed to the cascade, a portion of each constituent entered the cascade with the UF_6 while the balance remained in the cylinder. The split is assumed to be as shown in Table 2.2.5-1.

The barrier is assumed to contain essentially all of the TRU and was contaminated with Np, Pu, and ⁹⁹Tc at an estimated 0.24 ppm, 4 ppb, and 7 ppm, respectively, prior to FY 1967 and 0.09 ppm, 2.2 ppb, and 7 ppm, respectively, thereafter (Ref. 2) that was removed during the equipment change-out. This barrier was decontaminated in the X-705 large equipment tunnel, where essentially all of the TRU and ⁹⁹Tc was assumed to go into solution. This process removed essentially all Np and Pu from the PORTS cascade and approximately ½0f the ⁹⁹Tc introduced through FY 1959. The ⁹⁹Tc from the PPF is assumed to continue to absorb on cascade surfaces until it reaches equilibrium. No record of ⁹⁹Tc releases or its presence in the product were found to have occurred during this campaign.

3.1.2 Campaign #2

In Campaign #2, the barrier is again assumed to contain essentially all of the TRU and ⁹⁹Tc that was removed during the CIP/CUP and purge converter change-outs. The CIP/CUP change-out replaced equipment that contained the Np and Pu and a portion of the ⁹⁹Tc. The Np and Pu remain near the feed point, while ⁹⁹Tc may be found almost anywhere above the feed point. For this reason, only ⁹⁹Tc is shown as present in the purge converters changed-out. During this campaign, the barrier and other removed equipment were decontaminated. The TRU and ⁹⁹Tc are assumed to go into solution. The decontamination and processing of the decontamination solutions are assumed to have been performed in the same period as the equipment removal. All of the ⁹⁹Tc is assumed to go into the raffinate or traps. Ninety-nine percent of the Np and Pu are assumed to end up in the oxide produced.

Beginning with FY 1972, additional data on PPF became available and this was used to calculate the quantity of ⁹⁹Tc present. Starting with FY 1976, the product produced at PORTS is known to contain ⁹⁹Tc. An average of about 2% of the total amount of ⁹⁹Tc remaining in the cascade is estimated to be removed annually. It is estimated that a total of 1,585g of ⁹⁹Tc was removed in the product stream through March 1999. In FY 1975, quantities of ⁹⁹Tc that were detected in air/water releases to the environment were removed from PORTS inventory. The ⁹⁹Tc is shown as removed from PORTS inventory in the year the product is withdrawn from the cascade.

The HRT/SRT received in FY 1968 and 1969 was not all fed immediately. Therefore, the RU appears in the year-end inventory until fed. The TRU and ⁹⁹Tc fed to the cascade during this period is assumed to have been almost completely removed during the CIP/CUP program and/or purge converter change-out. No significant quantity of material is believed to have been removed during the 7A compressor change-out.

3.1.3 Campaigns #3 and #4

The UF₆ from the miscellaneous cascade feed and non-UF₆ scrap returns are summarized in the last two campaigns. The RU and contaminants are assumed to be fed or processed uniformly over the period from the earliest feed/processed date to the latest feed/processed date. Some of this material remains in storage.

3.2 Uranium Receipts

See Figure 3.1-1 and Table 3.2-1 for a summary of the RU received each FY and its source. A total of 1,123.7 MTU of RU (all forms) was received at PORTS. The table does not include Paducah or Oak Ridge product feed, which PORTS considers to be ⁹⁹Tc contaminated, but not RU. However, the mass flow includes the ⁹⁹Tc constituent of these PORTS feeds.

Table 3.2-1
PORTS Receipts Summary (RU Only)

		Net Weight (kgU)									
Shipping Facility	Uranium Form	FY 1955	FY 1956	FY 1957	FY 1958	FY 1966	FY 1967	FY 1968	FY 1969	FY 1972	
Allied Chemical	UO ₃										
Babcock & Wilcox	UF ₆										
Division of International	UF ₆							151			
Affairs	UNH					7	39				
Fernald	U_3O_8										
France	UF ₆									65	
Germany	UNH										
	UF ₄			865							
K-25	UF ₆		296, 504								
K-23	UO_2			418							
	UO_3			3,319							
NUMEC	UF ₆									330	
Paducah	UF ₆	105,873	54,649	6,156	64,311				567,620		
United Kingdom	UNH										
USAEC Office Safeguards & Materials. Management.	UF ₆								2,833		
Y-12	U_3O_8										
Grand Total		105,873	351,154	10,758	64,311	7	39	151	570,453	395	

Table 3.2-1 (Cont'd)

PORTS Receipts Summary (RU Only)

		Net Weight (kgU)								
Shipping Facility	Uranium Form	FY 1973	FY 1974	FY 1975	FY 1976	FY 1976.5	FY 1977	FY 1978	Grand Total	
Allied Chemical	UO ₃	1,376		1,403	1,295				4,074	
Babcock & Wilcox	UF ₆						153		153	
Division of International	UF ₆								151	
Affairs	UNH								46	
Fernald	U ₃ O ₈				7,798				7,798	
France	UF ₆	202	324	128	273	112	152	235	1,586	
Germany	UNH				6,860				6,860	
	UF ₄								865	
W 25	UF ₆								296,505	
K-25	UO_2								418	
	UO ₃								3,319	
NUMEC	UF ₆								330	
Paducah	UF ₆								798,609	
United Kingdom	UNH			7					7	
USAEC Office Safeguards & Materials. Management.	UF ₆								2,833	
Y-12	U_3O_8						104		104	
Grand Total		1,578	324	1,538	16,226	112	409	235	1,123,658	

3.3 Uranium Shipments

See Figure 3.1-1 and Table 3.3-1 for a summary by FY of the RU shipments from PORTS each FY and the receiving facility. PORTS shipped a total of 15.6 MTU of RU. The table does not include Paducah or Oak Ridge product feed, which PORTS considers to be ⁹⁹Tc contaminated, but not RU.

Table 3.3-1
PORTS Shipment Summary (RU Only)

Receiving Facility		Net Weight (kgU)								
	Uranium Form	FY 1955	FY 1956	FY 1972	FY 1974	FY 1982 - 1984	FY 1998	Grand Total		
B&W	UO_3						800	800		
France	UF ₆			65				65		
K-25	UF ₆		3,102					3,102		
NLO	U_3O_8					10,500		10,500		
Paducah	UF ₆	920	582		(368)			1,134		
Grand Total		920	3,684	65	(368)	10,500	800	15,601		

3.4 Recycled Uranium Waste

Central to the assumptions of this study is the concept of RU losing its identity through processing or treatment (i.e., the RU is blended with usually enormous amounts of non-RU resulting in product and tails streams containing deminimus quantities of RU). Wastes, therefore, are not classified as RU wastes, but rather wastes potentially contaminated with either TRU or FP. Such materials as alumina, NaF, and MgF₂ trapping media, contaminated pump oils, tower ash, and filter ash could constitute either waste or scrap depending upon the economics of processing and values of the recovered uranium. Holding pond and heavy metal sludges and ion exchange resins would constitute wastes from uranium recovery.

Quantification of the TRU/FP component of all of these streams could not be reliably accomplished within the time constraints of this report. Data on holding pond sludges have already been discussed. Data on filter ash have also been discussed. Some alumina and ion exchange resin data has been located, but not reviewed. NaF data remain to be discovered.

3.5 Recycled Uranium Scrap

For this study PORTS RU scrap is defined as RU scrap that was received from various sources either for conversion to UF₆ but was never converted to UF₆, or as RU-UF₆ feed that was never fed. Materials such as uranium heels in UF₆ cylinders that contained RU would meet this definition. There were 0.8 MTU of RU heels returned to PGDP and 0.8 MTU of RU heels returned to ORGDP. In addition, oxides (U₃O₈) produced from uranium recovery that contain TRU/FP could conceivably be considered RU scrap. In that regard, 0.85 MTU of highly enriched uranium oxides were shipped to BWXT during the HEU removal program. An unknown quantity of LEU oxides remain on site that potentially contain TRU/FP and may be considered scrap or waste depending upon the economics of processing and value of the recovered uranium.

3.6 Inventory as of March 31, 1999

A total of 8.3 MTU of RU (all forms) was in inventory at PORTS as of March 31, 1999. Table 3.6-1 shows the breakout by uranium form and includes the source of the material and the amount of uncertainty included in the inventory.

Table 3.6-1
PORTS March 31, 1999 Inventory of RU

Source Facility	Form	Amount Received (MTU)	Conv to UF ₆ (MTU)	Conv to U ₃ O ₈ (MTU)	UF ₆ Fed to Cascade (MTU)	Amount Shipped (MTU)	Amount in Inventory 03/31/99 (MTU)	Inventory Uncertainty 03/31/99 (MTU)
Allied (ICPP)	UO ₃	4.08	1.4		1.4	0.8	1.8	0.08
B&W	UF ₆	0.15			0.15			0
Div. of I.A.	UF ₆	0.15			0.15			0
DIV. OI I.A.	UNH	0.04						0.04
Fernald	U_3O_8	7.8	4.2		0.46	3.6	3.74	0
France	UF ₆	1.6			1.1	0.01		0.4
Germany	UNH	6.9		6.9		6.9		0
NUMEC	UF ₆	0.33			0.33			0
	UF ₄	0.86						0.86
ODCDD	UF ₆	296.5			293.4	3.1		0
ORGDP	UO ₂	0.4						0.4
	UO ₃	3.3						3.3
PGDP	UF ₆	798.6			797.5	1.1		0
USAEC	UF ₆	2.8			0.07		2.73	0
Y-12	U_3O_8	0.1						0.1
TOTAL		1123.61	5.6	6.9	1094.56	15.6	8.27	5.18